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LaSalle Generating Station
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July 24, 2000

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

LaSalle County Station, Unit 2
Facility Operating License No. NPF-18
NRC Docket No. 50-374

Subject: Licensee Event Report

In accordance with 10 CFR 50.73(a)(2)(iv), Commonwealth Edison (ComEd) Company is submitting Licensee Event Report #00-003-00, Docket No. 050-374.

Should you have any questions concerning this letter, please contact Mr. Frank A. Spangenberg, III, Regulatory Assurance Manager, at (815) 357-6761, extension 2383.

Respectfully,

A handwritten signature in black ink, appearing to read "Charles G. Pardee", is written over a horizontal line.

Charles G. Pardee
Site Vice President
LaSalle County Station

Attachments: Licensee Event Report

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - LaSalle County Station

IE22

LICENSEE EVENT REPORT (LER)

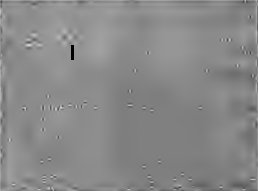


Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (T-6 F33), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1): LaSalle County Station, Unit 2

DOCKET NUMBER (2) 05000374

PAGE (3)
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TITLE (4) Scram on Low Reactor Water Level Due to Loss of 2A Turbine Driven Reactor Feed Pump Flow

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER		
06	22	00	00	003	00	07	24	00	FACILITY NAME	DOCKET NUMBER		
OPERATING MODE (9)		1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)		098										
			<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(2)(v)			<input type="checkbox"/> 50.73(a)(2)(i)			<input type="checkbox"/> 50.73(a)(2)(viii)		
			<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2003(a)(3)(i)			<input type="checkbox"/> 50.73(a)(2)(ii)			<input type="checkbox"/> 50.73(a)(2)(x)		
			<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 20.2003(a)(3)(ii)			<input type="checkbox"/> 50.73(a)(2)(iii)			<input type="checkbox"/> 73.71		
			<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 20.2003(a)(4)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			<input type="checkbox"/> OTHER		
			<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(1)			<input type="checkbox"/> 50.73(a)(2)(v)			Specify n Abstract below or in NRC Form 366A		
<input type="checkbox"/> 	<input type="checkbox"/> 50.36(c)(2)			<input type="checkbox"/> 50.73(a)(2)(vii)								
LICENSEE CONTACT FOR THIS LER (12)												
NAME Daniel Szumski, System Engineering								TELEPHONE NUMBER (Include Area Code) (815) 357-6761 Extension 2082				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)												
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX			CAUSE	SYSTEM	COMPONENT	REPORTABLE TO EPIX		
SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)				<input checked="" type="checkbox"/> NO								

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines 16)

At 0728 hours on June 22, 2000, LaSalle Unit 2 experienced an automatic reactor scram due to low reactor water level. The low reactor water level was caused by a loss of flow from the 2A turbine driven reactor feed pump (TDRFP).

The root cause of the event was the intrusion of wear products into the high pressure control oil pressure regulating relief valve (2FW199A). The wear products became lodged between the relief valve seat and disk, which allowed the oil system pressure to be relieved. This caused the 2A TDRFP control valve to close, and the loss of flow from the 2A TDRFP. The resulting reactor water level transient led to the automatic scram.

The corrective action was to install duplex oil filters on the 2A and 2B TDRFP high pressure control oil systems. The safety significance of this event was minimal. All control rods fully inserted and all systems responded as expected to the scram. The Emergency Core Cooling Systems were not challenged.

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PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor, 3489 Megawatts Thermal Rated Core Power

A. CONDITION PRIOR TO EVENT

Unit(s): 2 Event Date: 06/22/00 Event Time: 0728 Hours
Reactor Mode(s): 1 Power Level(s): 098
Mode(s) Name: Run

B. DESCRIPTION OF EVENT

At 0728 hours on June 22, 2000, LaSalle Unit 2 experienced an automatic reactor scram due to low reactor water level. The reactor operator (RO) received a 2A turbine driven reactor feed pump (TDRFP) speed control trouble alarm and observed the steam control valve going closed and reactor water level decreasing. The RO observed the 2B TDRFP responding as expected (flow increasing), while the 2A TDRFP flow continued to decrease and was not responsive to automatic or manual control inputs. An automatic reactor scram occurred at 12.5 inches reactor vessel water level.

The investigation into the event determined that the operating crew responded to the feedwater transient in accordance with procedures and management expectations. All systems responded as expected to the scram.

The cause of the event was found to be the intrusion of wear products into the high pressure control oil components used in the 2A TDRFP control system, which allowed the oil system pressure to be relieved and the TDRFP control valve to close. The loss of flow from the 2A TDRFP resulted in a reactor water level transient leading to the scram.

This event is reportable in accordance with 10 CFR 50.73(a)(2)(iv), as an event that resulted in a manual or automatic actuation of any engineered safety feature (ESF), including the reactor protection system (RPS).

C. CAUSE OF EVENT

The root cause of the event was the intrusion of wear products into the high pressure control oil pressure regulating relief valve (2FW199A) that were in excess of the valve's tolerances. The wear products became lodged between the valve seat and disk, allowing the high pressure control oil supply to drain. With an inadequate oil supply to the secondary operating cylinder of the 2A TDRFP control valve, the valve closed, resulting in the 2A TDRFP ramping down in speed (see Figure 1).

Wear products of sufficient size were found in the control oil system, although none was found on the seating surface of the 2FW199A valve. It is suspected that the material was flushed out of the valve during the transient.

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The corrective action was to install in-line duplex filters on the 2A and 2B TDRFP high pressure control oil system supply lines. Filters for the high pressure control oil had been previously installed on Unit 1, but had not been installed on Unit 2.

D. SAFETY ANALYSIS

The safety significance of this event was minimal. All control rods fully inserted and all systems responded as expected to the scram. The Emergency Core Cooling Systems were not challenged.

E. CORRECTIVE ACTIONS**Corrective Actions to Prevent Recurrence**

In-line duplex filters were installed on the 2A and 2B TDRFP high pressure control oil system supply lines (Complete).

F. PREVIOUS OCCURRENCES

LER Number	Title
374/99-002	Automatic Scram Due to Failure of Reactor Water Level Control

On August 21, 1999, an automatic scram of Unit 2 occurred due to low reactor water level, due to a failure of the 2A turbine driven reactor feed pump. The cause of the event was determined to be a failure of the 2A TDRFP servo valve (2FW163AA), along with a failure of the control room crew to properly respond to a reactor level transient. The root cause and corrective actions would not have prevented this event.

G. COMPONENT FAILURE DATA

Since no component failure occurred, this section is not applicable.

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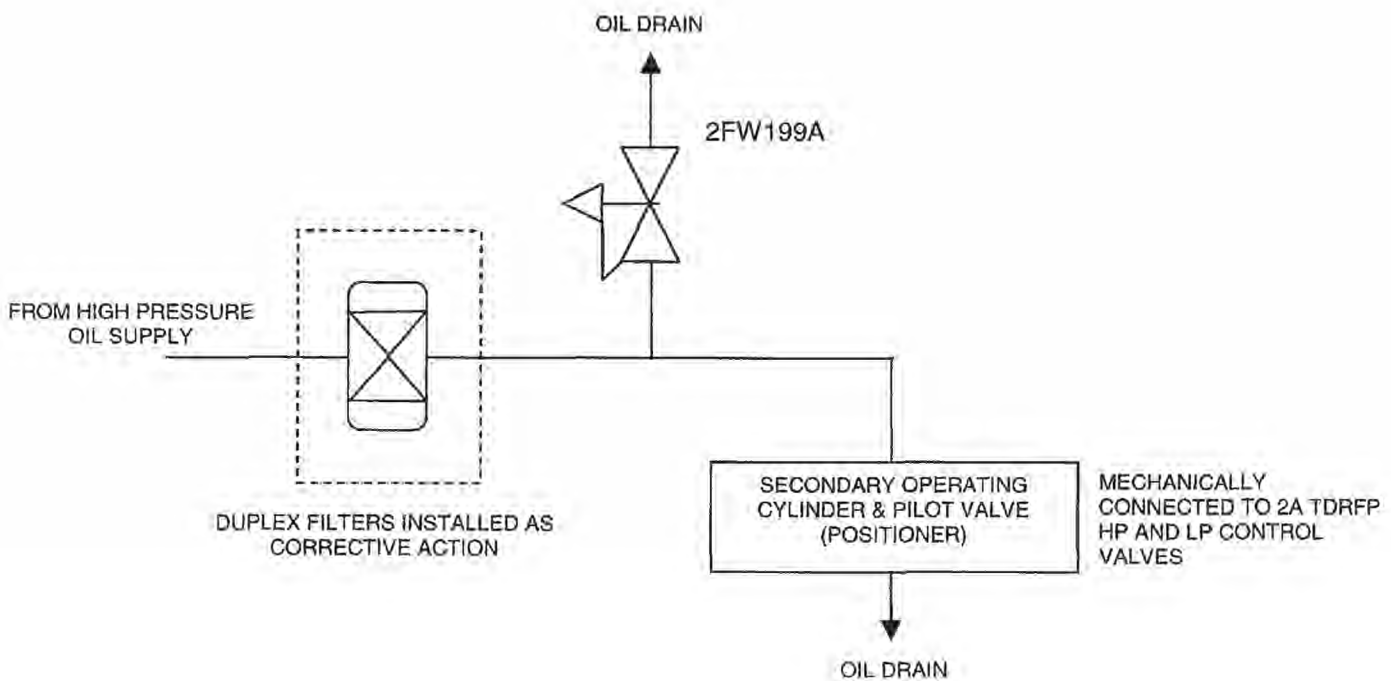


Figure 1